

# Seminar

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## Institute for Plasma Research

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**Title:** Cross section Measurement of Tin and Rhenium Isotopes with Covariance Analysis and Simulation of Neutron Response Function for NE213 Scintillation Detector

**Speaker:** Dr. Zara Aftab  
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**Date:** 17<sup>th</sup> April 2026 (Friday)

**Time:** 10:30 A.M.

**Venue:** Seminar Hall, IPR

**Join the talk online:** URL: <https://bharatvc.nic.in/viewer/5992138016>

(Conference ID: 5992138016; Password: 232142)

### Abstract

Accurate knowledge of neutron-induced reaction cross sections is crucial for nuclear technology applications, including reactor design, radiation shielding, and nuclear data evaluation [1]. Despite the availability of evaluated data files and theoretical predictions, significant discrepancies still exist between different sources. Therefore, the experimental determination of cross sections under controlled conditions remains indispensable. These data are fundamental to reactor physics calculations, such as neutron transport, radioisotope production, and safety assessments. In this study, neutron-induced reaction cross sections for selected isotopes of fusion-relevant materials (Tin and Rhenium [2-5]) were measured for 15.04 MeV and 15.02 MeV neutrons, respectively. Specifically, the cross sections for  $^{117}\text{Sn}(n,p)^{117}\text{gIn}$ ,  $^{118}\text{Sn}(n,\alpha)^{115}\text{gCd}$ ,  $^{120}\text{Sn}(n,\alpha)^{117}\text{gCd}$ ,  $^{185}\text{Re}(n,2n)^{186}\text{gRe}$ ,  $^{187}\text{Re}(n,2n)^{186}\text{gRe}$ ,  $^{187}\text{Re}(n,p)^{187}\text{W}$ ,  $^{187}\text{Re}(n,\alpha)^{187}\text{Ta}$  reactions were determined using the neutron activation technique followed by  $\gamma$ -ray spectroscopy. To ensure the reliability of the results, all required correction factors and parameter uncertainties were considered, and a detailed uncertainty analysis was performed using the covariance method. The experimental data were utilized to optimize theoretical model parameters using the TALYS-2.0 and TASMANT-2.0 codes. Furthermore, a sensitivity analysis was conducted to identify the key theoretical model parameters influencing the cross sections.

Another task focuses on the simulation of neutron response function of NE213 detector. The neutron response function relates the measured spectrum of the detector to the incident neutron energy spectrum. In this work, neutron response function of an NE213 scintillation detector was simulated using MCNP which is required for the unfolding of the neutron spectra.

### References:

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  - [5] S. Nogami, D. Terentyev, A. Zinovev, C. Yin, M. Rieth, G. Pintsuk, and A. Hasegawa, Neutron irradiation tolerance of potassium-doped and rhenium-alloyed tungsten, *J. Nucl. Mater.* **553**, 153009 (2021)
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